

**22.251 Systems Analysis of the Nuclear Fuel Cycle  
Fall 2009**

**Laboratory Exercise #2: CASMO Assembly Calculations**

Using the whole-assembly CASMO-4 PWR input given in class, answer the following:

- a) Calculate and plot  $\rho$  as a function of burnup up to 60 MWd/kg with and without the burnable poison (Gd). Estimate the residual poison  $\Delta\rho$  at high burnup (i.e. 60 MWd/kg).
- b) At zero burnup calculate the following reactivity feedback coefficients for gadolinium-poisoned fuel assemblies:
  - 1) Moderator Temperature Coefficient (MTC) in units of  $\Delta\rho$  per K by perturbing the temperature of the moderator by 5 K around the reference value. Plot the MTC as a function of coolant temperature in the range between 400 K and 600 K for three different boron concentrations in the coolant: 0 ppm, 1000 ppm, and 2000 ppm. Plot the three MTC vs. temperature curves on the same graph and qualitatively explain the behavior of the curves.
  - 2) Fuel Temperature Coefficient (FTC or Doppler coefficient) in units of  $\Delta\rho$  per K by reducing the reference average fuel temperature to 800 K.
  - 3) Void Coefficient (VC) in units of  $\Delta\rho$  per %void by reducing the moderator density by 10%.
- c) Calculate the following additional effects:

Create your own “reactivity ladder” such as attached on the following page by simulating:

- 1)  $\Delta\rho$  from cold zero power (CZP) to hot zero power (HZP) between 30°C and 300°C with isothermal moderator and fuel.
- 2)  $\Delta\rho$  from hot zero power to hot full power (HFP)
- 3)  $\Delta\rho$  due to xenon buildup at HFP by burning for 100 effective full power hours (EFPH)
- 4) Calculate the control rod worth by finding the  $\Delta\rho$  between water filled and Ag-In-Cd (AIC) rod filled guide tubes. Assuming that only 53 fuel assemblies out of 193 in a typical PWR core have control rods, discuss the factors that will affect the individual control rod worth in the full core.

Note: In all parts (except part a) assume poisoned fuel assemblies.

# Approximate Reactivity Ladder for LWR

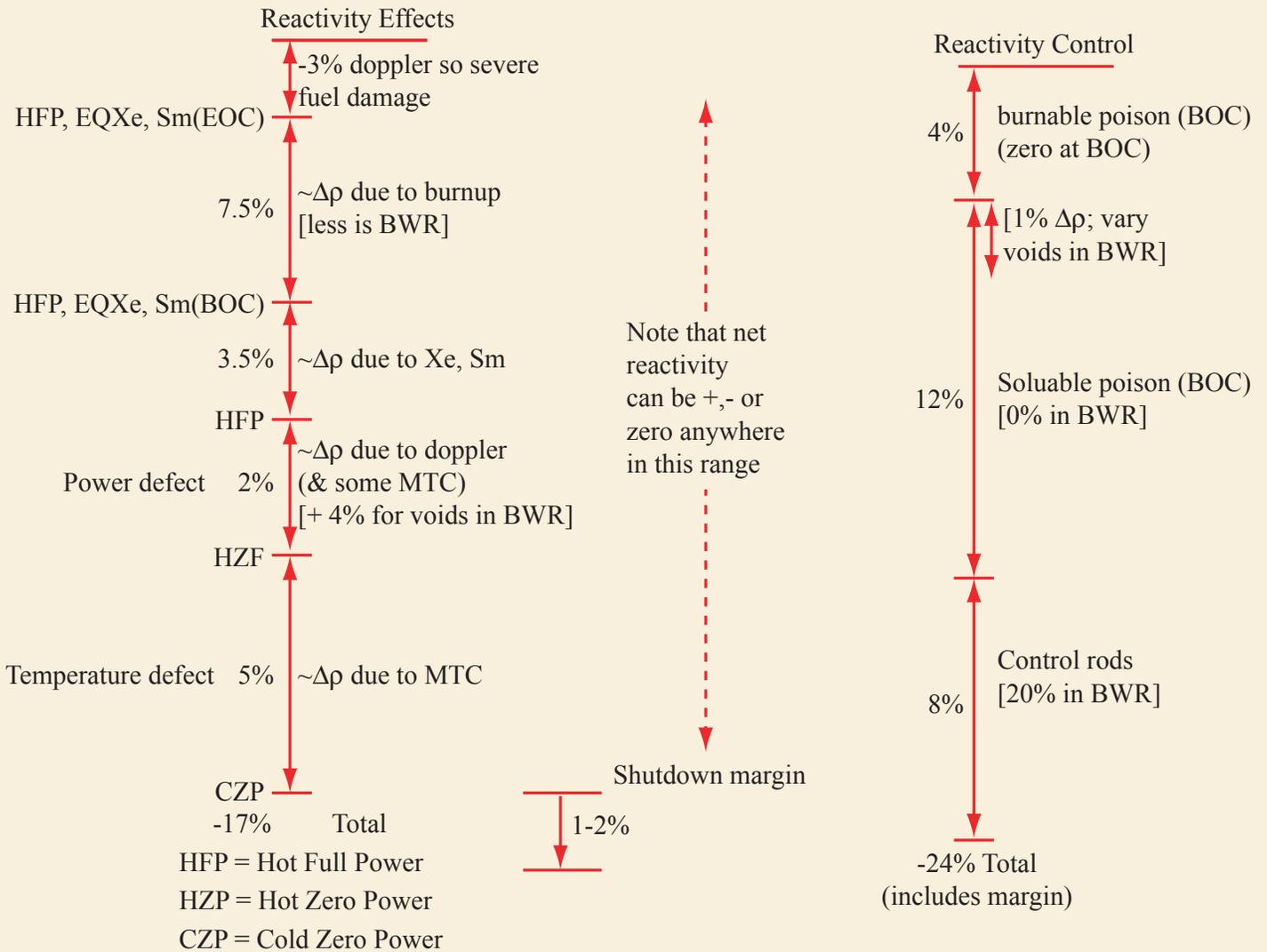


Image by MIT OpenCourseWare.

MIT OpenCourseWare  
<http://ocw.mit.edu>

22.251 Systems Analysis of the Nuclear Fuel Cycle  
Fall 2009

For information about citing these materials or our Terms of Use, visit: <http://ocw.mit.edu/terms>.