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SUSTAINABLE ENERGY

2.650J/10.291J/22.081J

**INTRODUCTION TO
SUSTAINABLE ENERGY**

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NUCLEAR WASTES AND YUCCA MOUNTAIN

NUCLEAR WASTE



Locations of Spent Nuclear Fuel and High-Level Radioactive Waste



Defense Complex Clean-Up

Support of Nonproliferation Initiatives, e.g. Disposal of DOE Foreign Research Reactor Spent Fuel

Commercial Spent Nuclear Fuel

Disposition of Naval Reactor Spent Nuclear Fuel

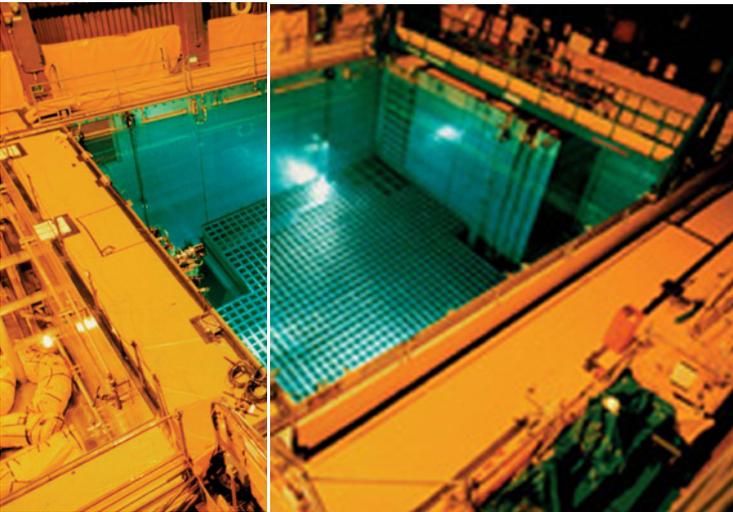
Source: *The Safety of a Repository at Yucca Mountain*, USDOE, CRWM, June 2008.



SPENT NUCLEAR FUEL

- 39 states with nuclear waste
- Five DOE sites with nuclear waste

Spent-fuel pools



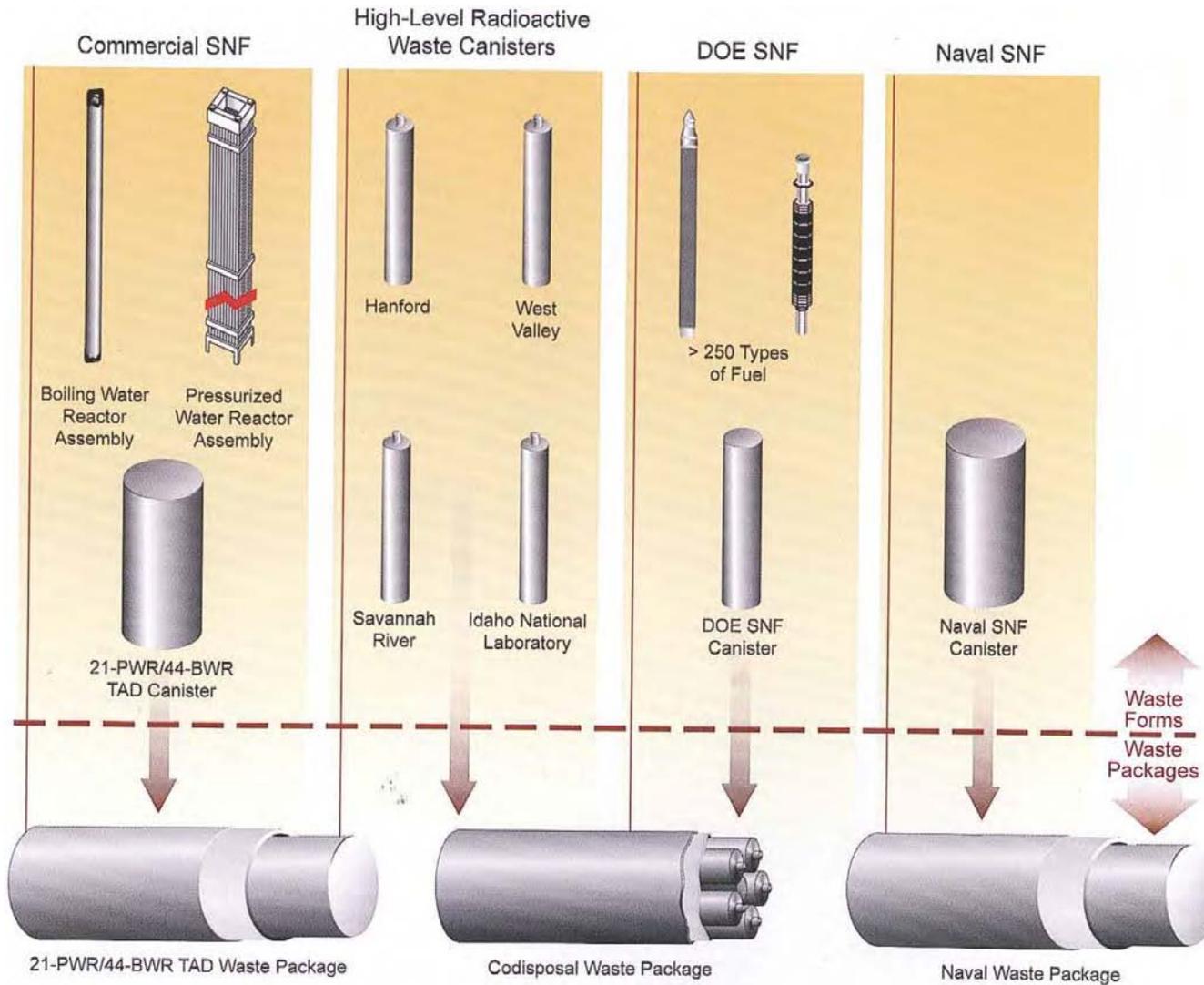
Dry cask storage



Photos of spent fuel pool and dry cask storage from the U.S. Nuclear Regulatory Commission.



WASTE FORMS AND PACKAGES



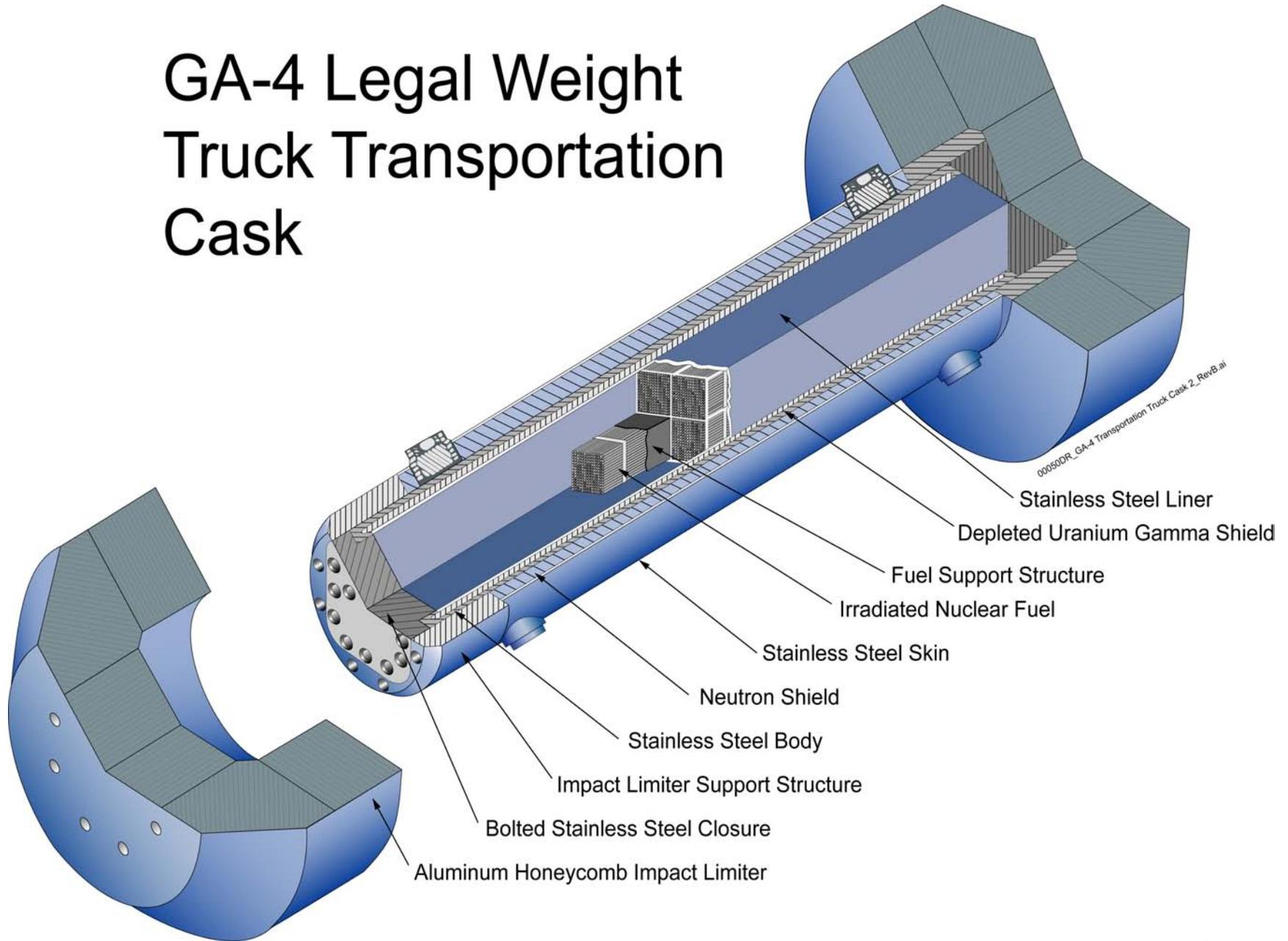
Drawing Not To Scale
06275PD_002.ai

Source: *The Safety of a Repository at Yucca Mountain*, USDOE, CRWM, June 2008.



TRANSPORTATION CASK

GA-4 Legal Weight Truck Transportation Cask





TIMELINE FOR NUCLEAR WASTE DISPOSAL

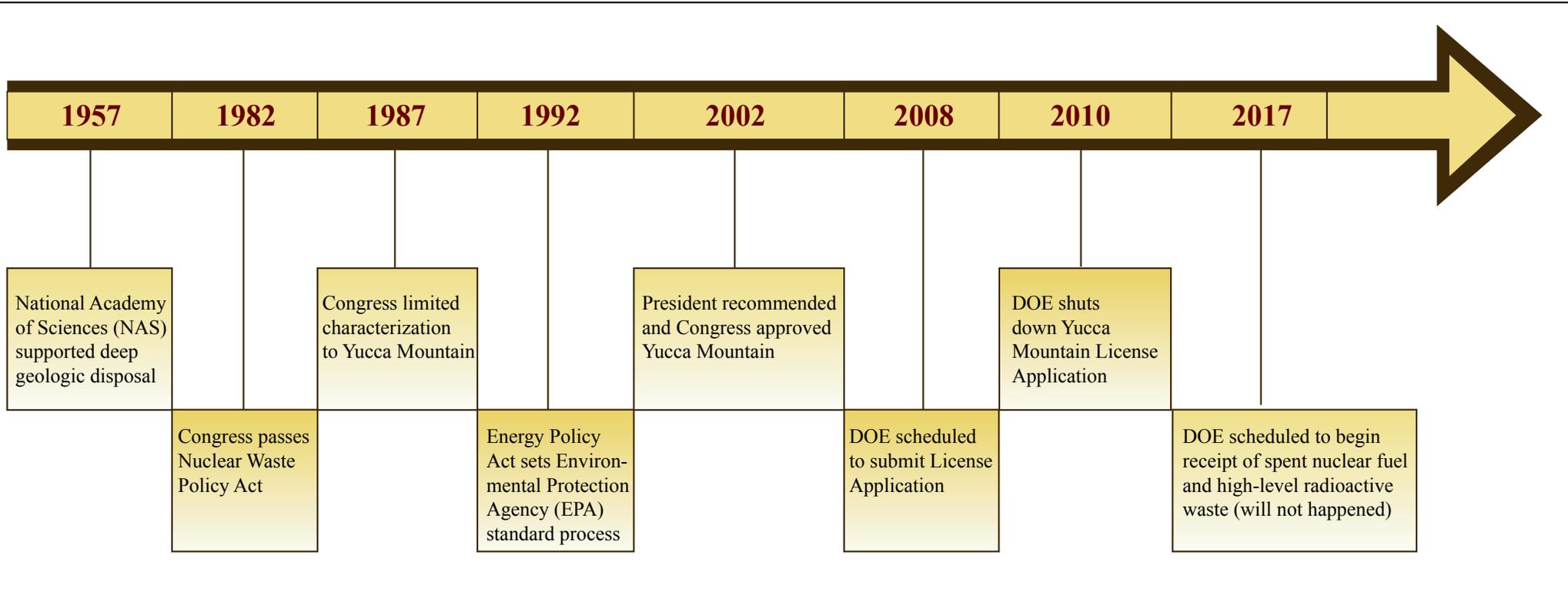


Image by MIT OpenCourseWare.



YUCCA MOUNTAIN, NEVADA

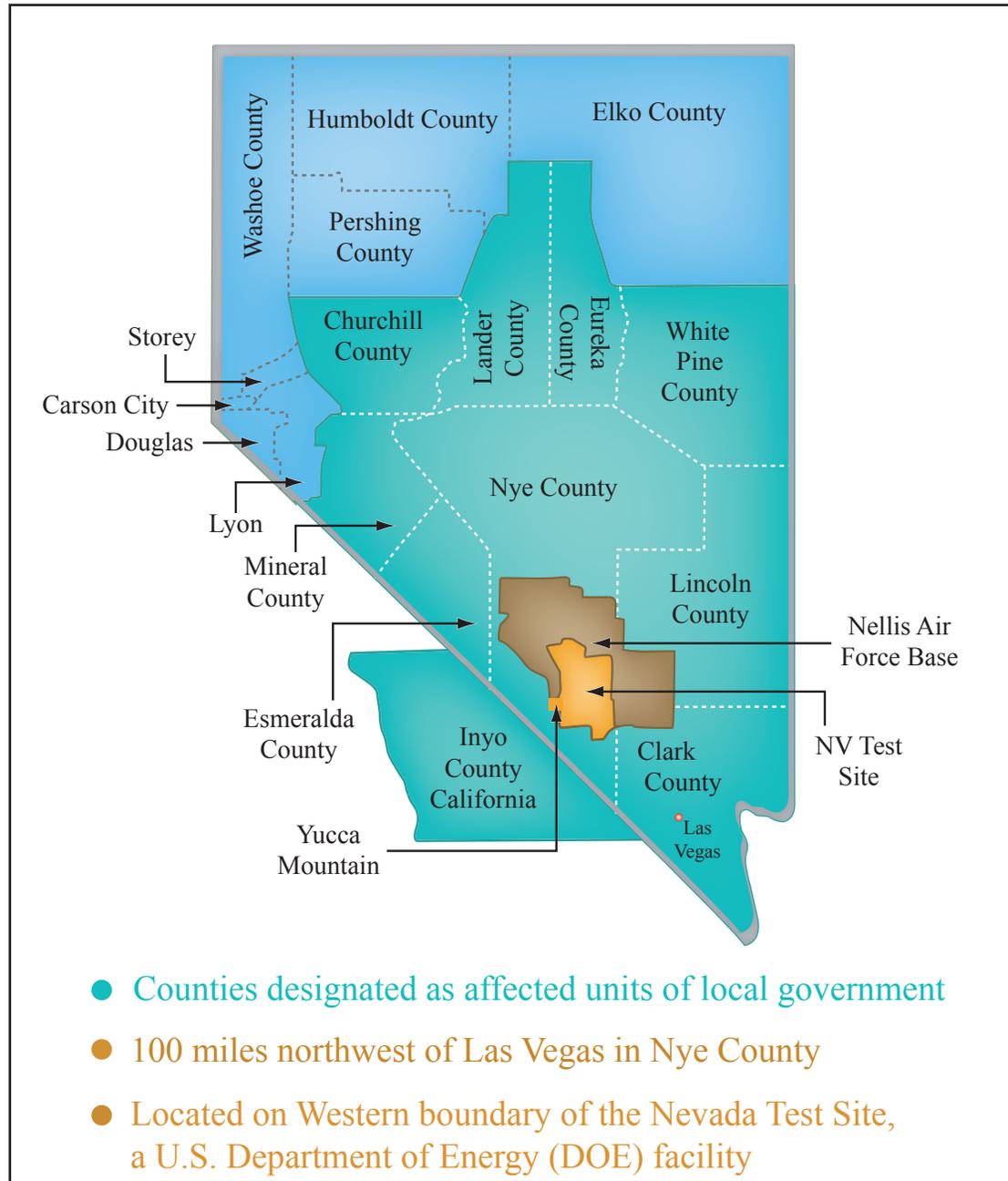


Image by MIT OpenCourseWare.



YUCCA MOUNTAIN SITE



Source: *The Safety of a Repository at Yucca Mountain*, USDOE, CRWM, June 2008.



YUCCA MOUNTAIN SUBSURFACE OVERVIEW

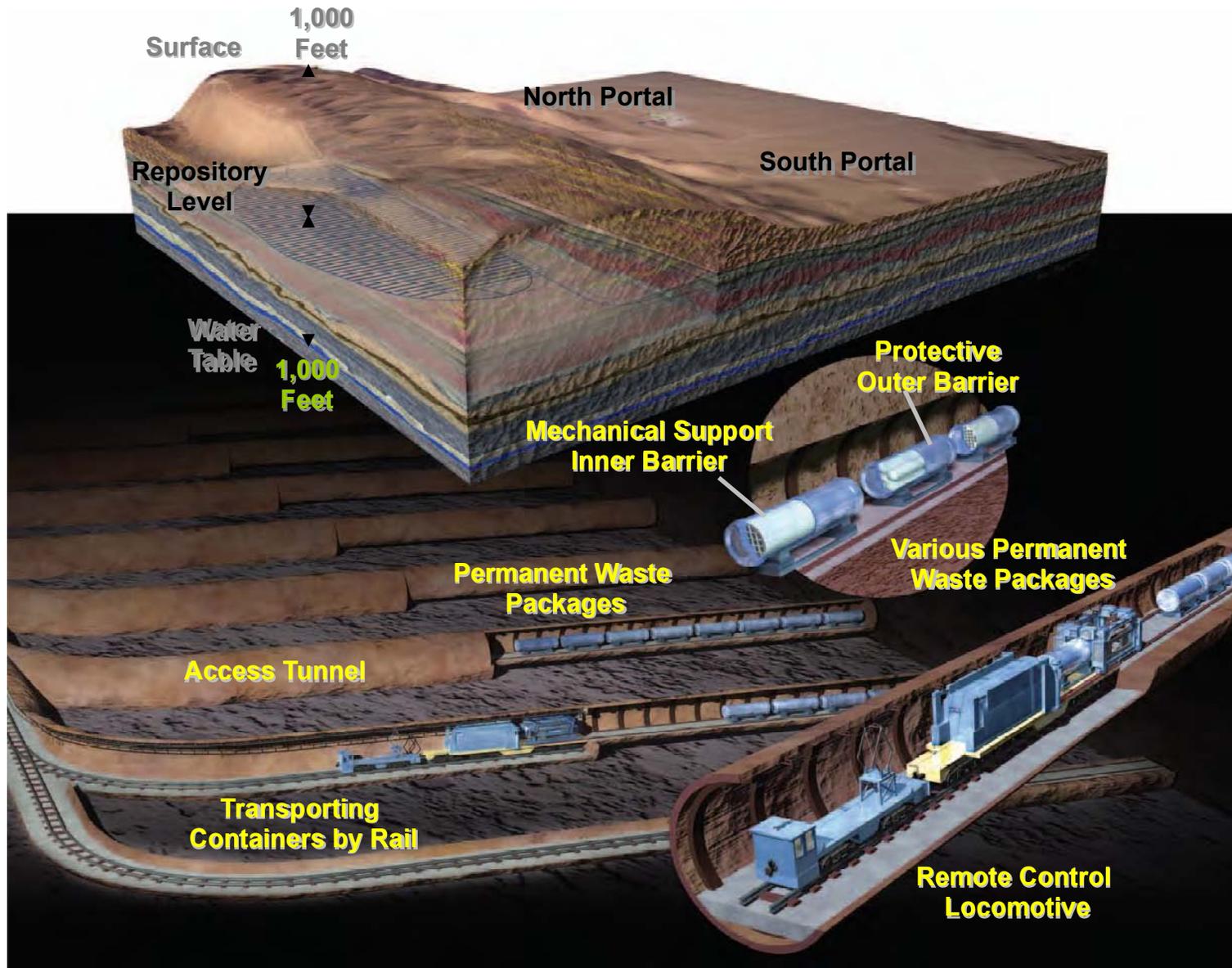
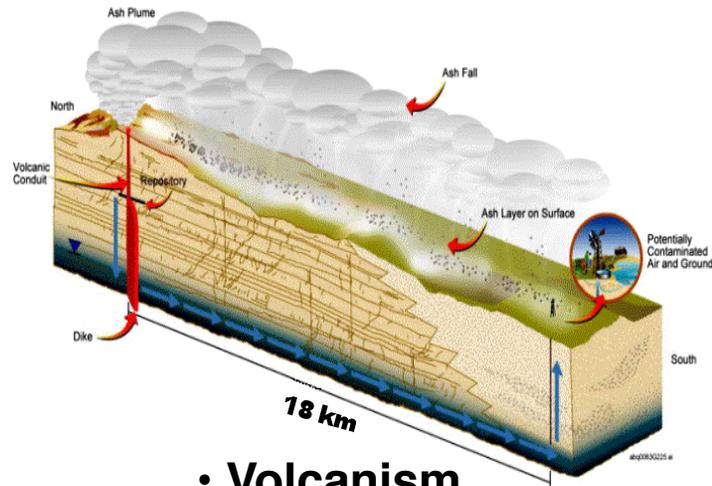


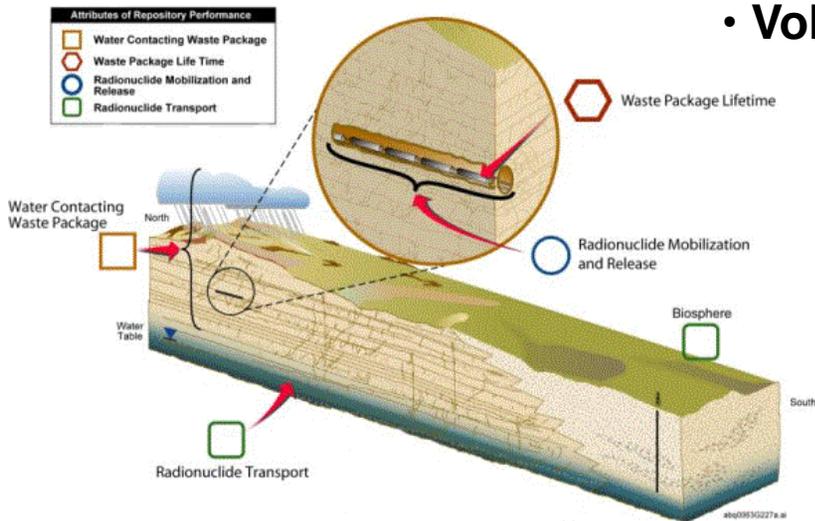
Image by U.S. Office of Civilian Radioactive Waste Management.



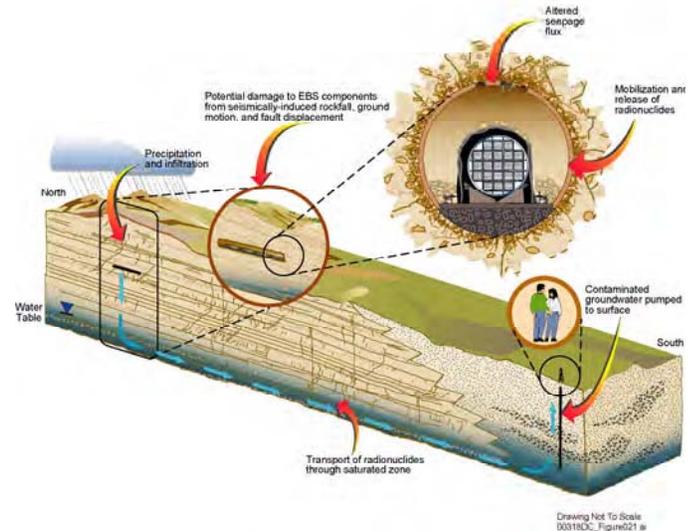
HYPOTHETICAL SCENARIOS



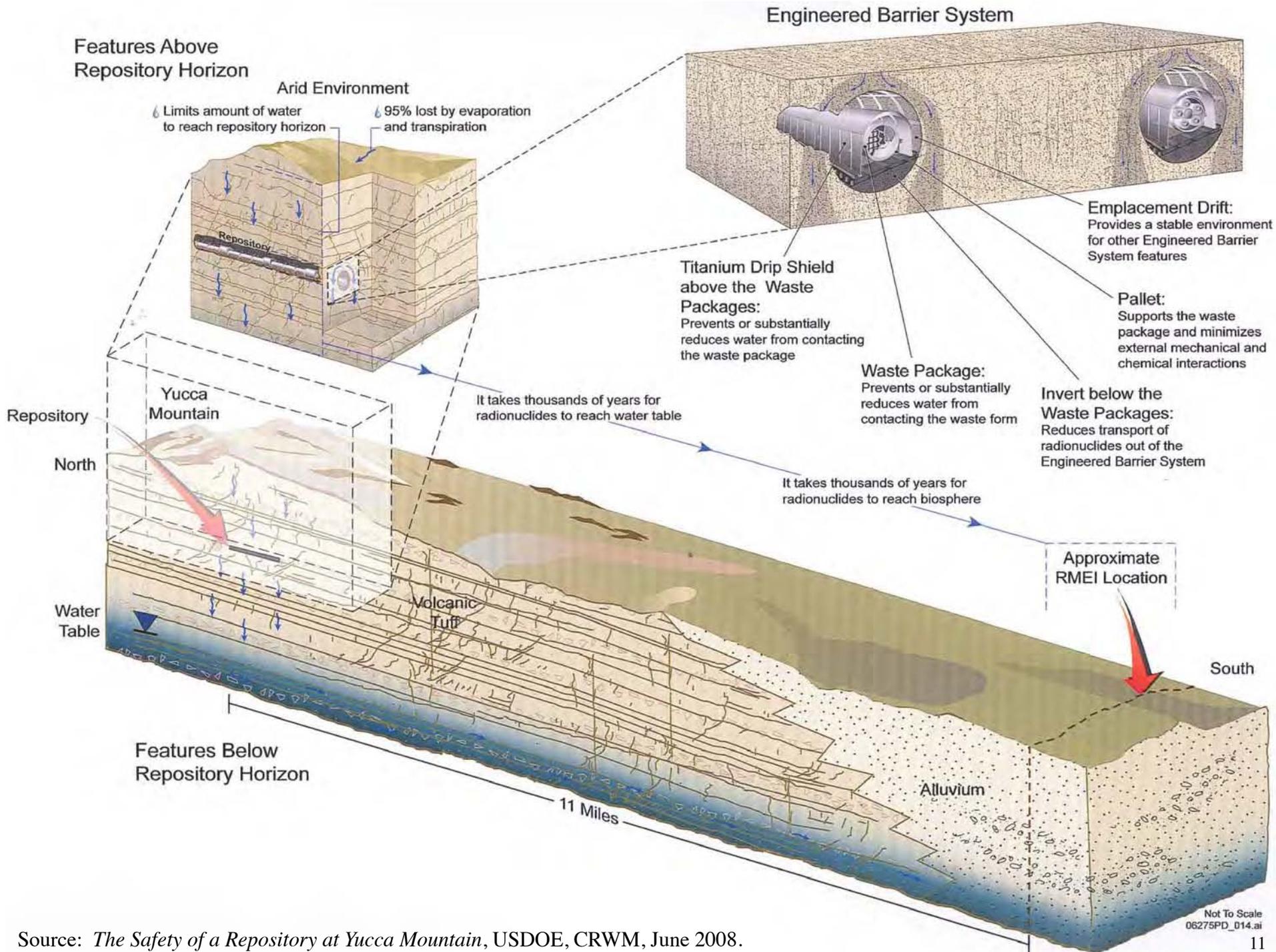
• Volcanism



- Nominal
- Early defects



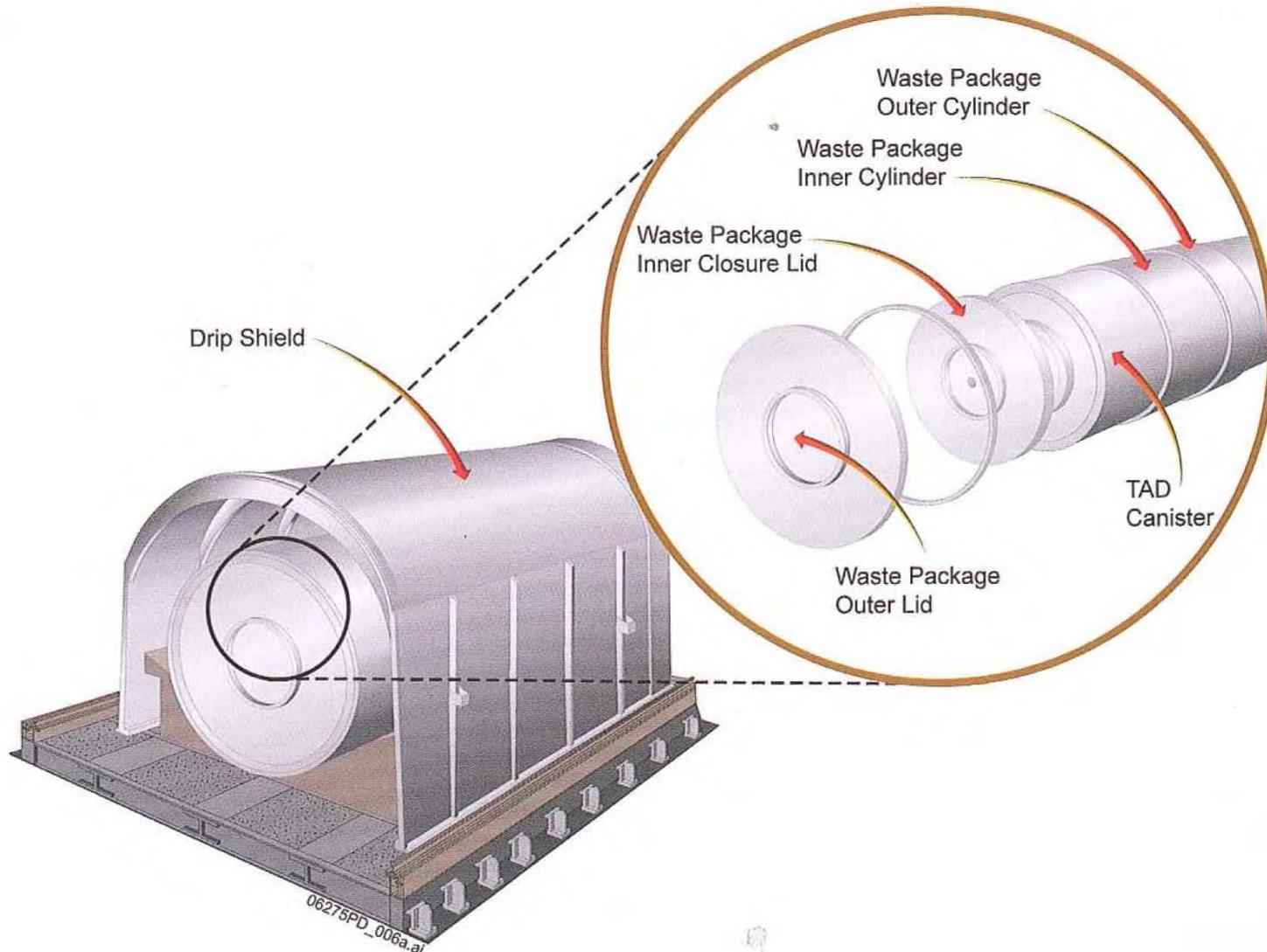
• Seismic



Source: *The Safety of a Repository at Yucca Mountain*, USDOE, CRWM, June 2008.



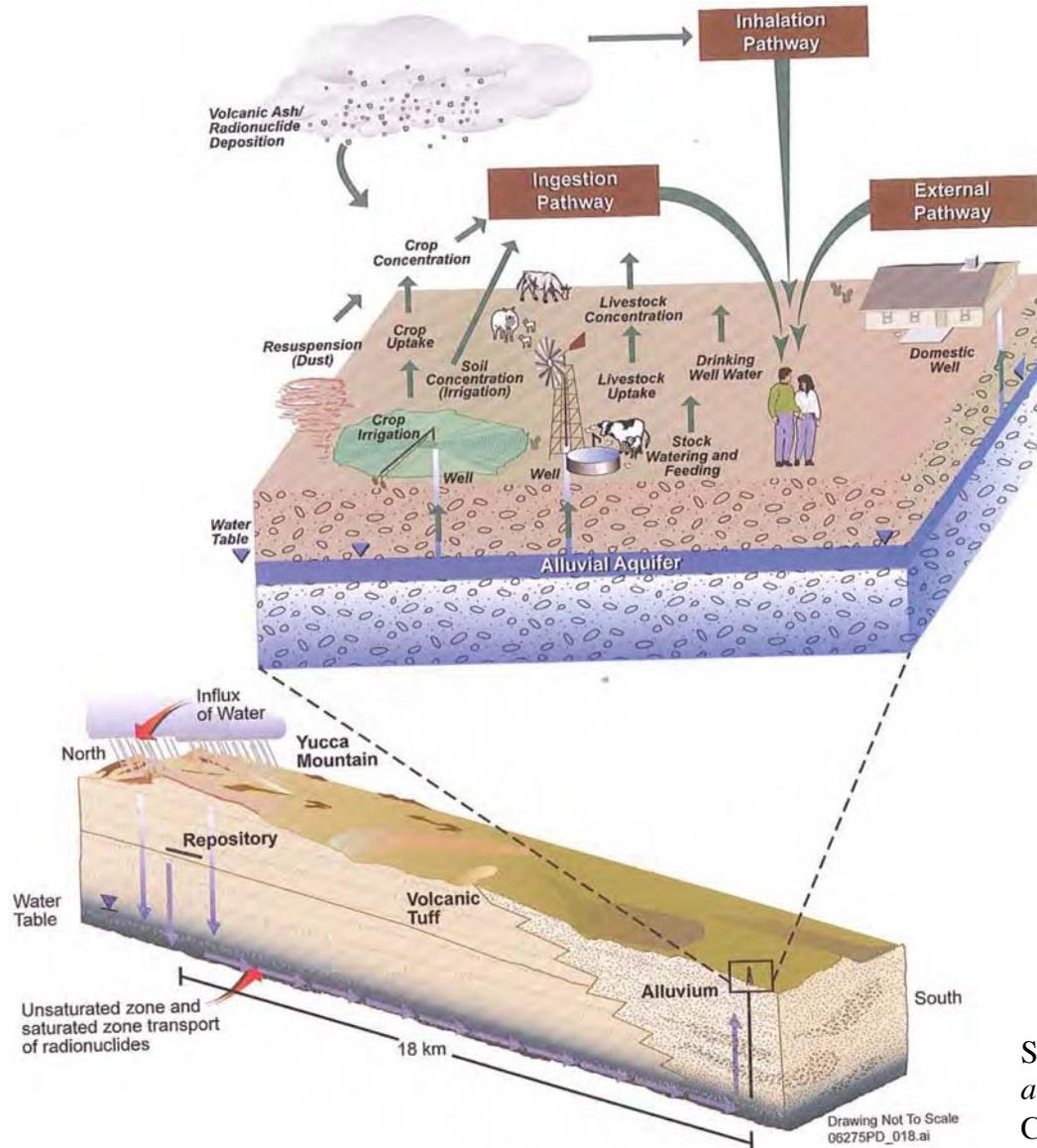
CANISTER PLACED INSIDE WASTE PACKAGE



Source: *The Safety of a Repository at Yucca Mountain*, USDOE, CRWM, June 2008.



LOCATION AND CHARACTERISTICS OF REASONABLE MAXIMALLY EXPOSED INDIVIDUAL AND FEATURES OF NATURAL SYSTEM BELOW REPOSITORY THAT LIMIT MOVEMENT OF RADIONUCLIDES TO THAT LOCATION



Source: *The Safety of a Repository at Yucca Mountain*, USDOE, CRWM, June 2008.



Yucca Mountain: Predicted average annual dose for 10,000 years

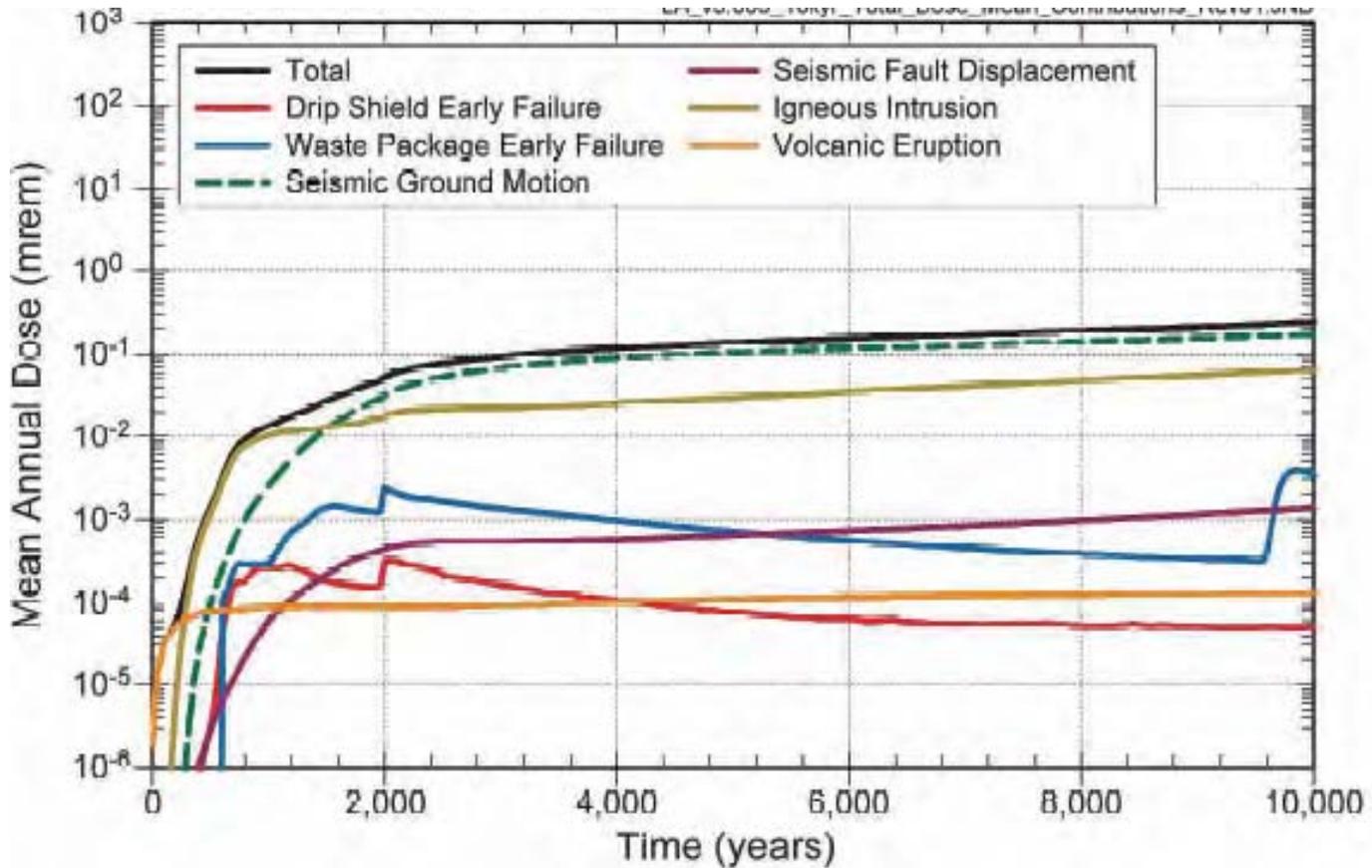


Fig. F-17 in *Draft Supplemental Environmental Impact Statement for a Geologic Repository at Yucca Mountain*. U.S. Department of Energy, October 2007, DOE/EIS-0250F-S1D.



Yucca Mountain: Predicted median annual dose for 1,000,000 years

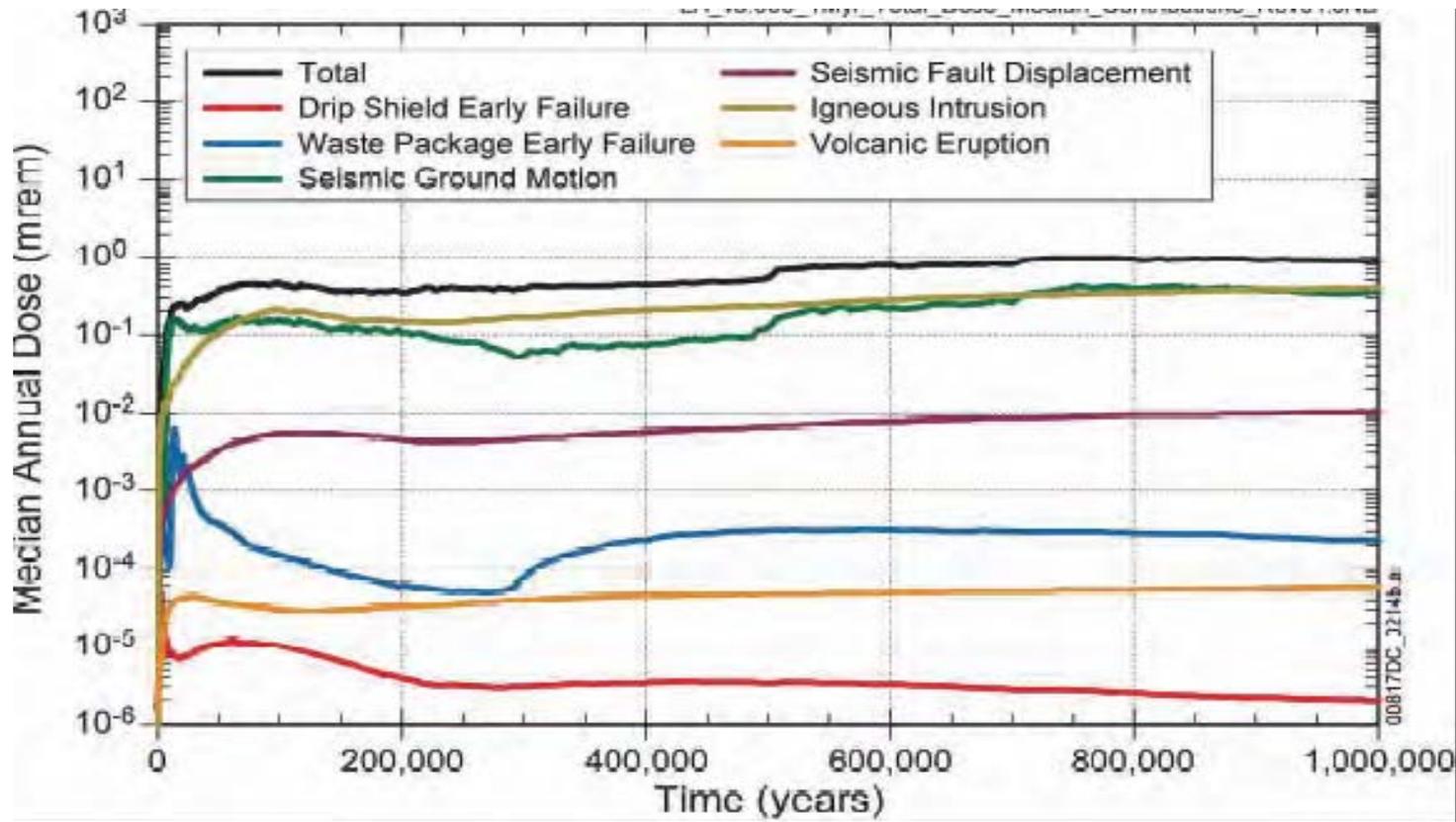


Fig. F-17 in *Draft Supplemental Environmental Impact Statement for a Geologic Repository at Yucca Mountain*. U.S. Department of Energy, October 2007, DOE/EIS-0250F-S1D.



POSTCLOSURE PERFORMANCE RESULTS

	Mean	Median	95th Percentile
Individual Protection Standard	No more than 15 mrem	No more than 350 mrem ^a	None (95th percentile is provided here for comparison only)
First 10,000 years	0.24 mrem		0.67 mrem
Time of occurrence	10,000 years		10,000 years
Post-10,000 years ^b		0.9 mrem	9.1 mrem
Time of occurrence		~800,000 years	1,000,000 years
Human Intrusion Standard		No more than 350 mrem ^a	
Post-10,000 years ^b		0.01 mrem	
Time of occurrence		~202,000 years	

NOTE: ^aProposed

^bWithin the proposed period of geologic stability, defined as 1 million years.

	Combined ²²⁶ Ra and ²²⁸ Ra Concentration	Gross Alpha Activity Concentration	Dose from Combined Beta- and Photon-Emitting Radionuclides
Groundwater Protection Standard: Limit for Activity Concentration or Annual Dose	5 pCi/L	15 pCi/L	4 mrem
Performance Results: Projected Maximum Mean Activity Concentration or Annual Dose	<10 ⁻⁶ pCi/L	10 ⁻⁴ pCi/L	Whole body: ~0.06 mrem Thyroid: ~0.26 mrem
Natural Background Level	0.5 pCi/L	0.5 pCi/L	Background level excluded in regulatory requirement

Source: *The Safety of a Repository at Yucca Mountain*, USDOE, CRWM, June 2008.



NUCLIDES OF INTEREST

Summary of Activities (Ci/MTU) for Radionuclides of Interest as a Function of Post-Irradiation Time

(NOTE: Nuclides whose contribution to the specific activity of the fuel is less than 0.001 Ci/MTU at a post-irradiation time of 150 days are left blank.)

Power = 30MW; Burnup = 33,000 MWD; Flux = 2.92×10^{13} n/cm²-sec

Z	Radionuclide	Half-life*	Discharge**	Curies/Metric Ton Uranium									
				160 days**	1 year**	10 years**							
1	H-3	12.26y	7.09E02	6.91E02	6.7E02	4.03E02	40	Zr-93	9.5x10 ⁵ y	1.89E00	1.89E00	1.89E00	1.89E00
4	Be-10	2.7x10 ⁶ y	-	-	-	-		Zr-95	65d	1.37E06	2.49E05	2.80E04	1.67E-11
6	C-14	5770y	-	-	-	-	41	Nb-93m	3.7y	1.45E-01	1.83E-01	2.31E-01	8.40E-01
20	Ca-41	1.1x10 ⁵ y	-	-	-	-		Nb-95	35d	1.38E06	4.73E05	5.95E04	3.61E-11
27	Co-60	5.27y	-	-	-	-	42	Mo-93	10 ⁴ y	-	-	-	-
28	Ni-59	8x10 ⁴ y	-	-	-	-	43	Tc-99	2.1x10 ⁵ y	1.43E01	1.43E01	1.43E01	1.43E01
34	Se-79	7x10 ⁴ y	3.98E-01	3.98E-01	3.98E-01	3.98E-01	44	Ru-103	40d	1.22E06	7.41E04	2.05E03	2.09E-22
36	Kr-85	10.4y	1.13E04	1.10E04	1.06E04	5.96E03		Ru-106	1.0y	5.45E05	4.03E05	2.73E05	5.50E02
37	Rb-87	4.7x10 ¹⁰ y	-	-	-	-	46	Pd-107	7x10 ⁶ y	1.10E-01	1.10E-01	1.10E-01	1.10E-01
38	SR-89	50.4d	7.18E05	8.51E04	5.53E03	5.15E-16	47	Ag-110m	249d	3.68E03	2.37E03	1.35E03	1.66E-01
	SR-90	28y	7.76E04	7.68E04	7.57E04	6.07E04	50	Sn-126	10 ⁵ y	5.46E-01	5.46E-01	5.46E-01	5.46E-01
39	Y-90	64.2h	8.07E04	7.68E04	7.58E04	6.07E04	51	Sb-125	2.0y	8.7E03	7.89E03	6.83E03	6.78E02
	Y-91	57.5d	9.38E05	1.43E05	1.28E04	1.89E-13	52	Te-127m	105d	1.54E04	5.77E03	1.57E03	1.30E-06
								Te-129	67.3m	3.37E05	1.42E03	2.17E01	0.0
							53	I-129	1.72x10 ⁷ y	3.71E-02	3.74E-02	3.74E-02	3.74E-02
								I-131	8.05d	8.61E05	9.23E-01	1.99E-08	0.0
							55	Cs-134	2.19y	2.46E05	2.12E05	1.76E05	8.38E03
								Cs-135	2.0x10 ⁶ y	2.86E-01	2.86E-01	2.86E-01	2.86E-01
								Cs-137	30y	1.08E05	1.07E05	1.05E05	8.56E04

Appendix A in Bishop, William P., and Frank J. Miraglia, Jr. *Environmental Survey of the Reprocessing and Waste Management Portions of the LWR Fuel Cycle*. U.S. Nuclear Regulatory Commission, October 1976, NUREG-0116/WASH-1248 Supplement 1.



NUCLIDES OF INTEREST, cont'

Z	Radionuclide	Half-life	Discharge	160 days	1 year	10 years
58	Ce-144	285d	1.11E06	7.52E05	4.56E05	1.51E02
61	Pm-147	2.5y	1.02E05	9.73E04	8.39E04	7.75E03
63	Eu-154	16y	6.99E03	6.86E03	6.69E03	4.53E03
	Eu-155	1.7y	7.48E03	6.33E03	5.11E03	1.63E02
82	Pb-210	21y				
84	Po-210	138d				
86	Rn-220	51.5s	1.50E-03	2.29E-03	3.40E-03	1.60E-02
	Rn-222	3.82d				
88	Ra-224	3.64d	1.50E-03	2.29E-03	3.40E-03	1.60E-02
	Ra-225	14.8d				
	Ra-226	1622y				
ACTINIDES						
90	Th-228	1.91y	1.49E-03	2.27E-03	3.38E-03	1.60E-02
	Th-229	7340y				
	Th-230	80y				
	Th-232	14y				
	Th-234	24.1d	3.14E-01	3.14E-01	3.14E-01	3.14E-01
91	Pa-226	1.8m				
92	U-233	1.62x10 ⁵ y				
	U-234	2.48x10 ⁵ y	7.51E-01	7.55E-01	7.59E-01	8.30E-01
	U-235	7.13x10 ⁸ y	1.71E-02	1.71E-02	1.71E-02	1.71E-02
	U-236	2.39x10 ⁷ y	2.88E-01	2.88E-01	2.88E-01	2.88E-01
	U-238	4.51x10 ⁹ y	3.14E-01	3.14E-01	3.14E-01	3.14E-01

TRU

TRU					
93	Np-237	2.2x10 ⁶ y	3.33E-01	3.40E-01	3.40E-01 3.43E-01
94	Pu-236	285y	3.50E-01	3.16E-01	2.75E-01 3.08E-02
	Pu-238	89y	2.72E03	2.82E03	2.86E03 2.70E03
	Pu-239	2.44x10 ⁴ y	3.18E02	3.23E02	3.23E02 3.23E02
	Pu-240	6.58x10 ³ y	4.77E02	4.77E02	4.77E02 4.79E02
	Pu-241	13y	1.05E05	1.03E05	1.00E05 6.53E04
	Pu-242	3.79x10 ⁵ y	1.38E00	1.38E00	1.38E00 1.38E00
	Pu-244	7.6x10 ⁷ y			
95	Am-241	458y	8.59E01	1.59E02	2.50E02 1.41E03
	Am-242	16h	6.34E04	9.14E00	9.12E00 8.75E00
	Am-243	7.95x10 ³ y	1.81E01	1.82E01	1.82E01 1.82E01
96	Cm-242	163d	3.34E04	1.70E04	7.12E03 7.18E00
	Cm-243	35y	3.71E00	3.68E00	3.68E00 2.99E00
	Cm-244	17.6y	2.44E03	2.40E03	2.35E03 1.67E03
	Cm-247	4x10 ⁷ y			

* Handbook of Chemistry and Physics, 45th Edition, Section 8

** ORNL-4628, "ORIGEN-The ORNL Isotope Generation and Depletion Code", May 1973, Tables A-7 and A-9.

Appendix A in Bishop, William P., and Frank J. Miraglia, Jr. *Environmental Survey of the Reprocessing and Waste Management Portions of the LWR Fuel Cycle*. U.S. Nuclear Regulatory Commission, October 1976, NUREG-0116/WASH-1248 Supplement 1.



BUILDUP OF REACTION PRODUCTS

Images removed due to copyright restrictions.

Please see Fig. 1, 2, 9-11 in Cohen, Bernard L. "The Disposal of Radioactive Wastes from Fission Reactors." *Scientific American* 236 (June 1977): 21-31.



DISPOSAL OPTIONS

- Sub-Seabed
- Ice Sheets
- Space
- Deep Bore Holes
- Geologic repositories for storing highly radioactive materials have been chosen by the National Academy of Science in several assessments versus the alternative means of storage or disposal of highly radioactive materials.

Source: S.A. Simonson, "Waste Technology Issues," undated.



TECHNICAL PERFORMANCE CRITERIA

- The bases for determining the performance of a geologic repository are established by regulations
- The regulations establish numerical release limits that are presumed to be
 - 1) Self consistent between regulating agencies
 - NRC within 5 km of repository
 - EPA beyond 5 km
 - 2) Based upon equivalency of different radionuclide risks with regard to dose to man
 - 3) Consistent with other societal risks
 - Current basis is indirectly related to demonstrating a total system performance probability of less than one chance in 10 of causing 1000 excess deaths per 10,000 years

Source: S.A. Simonson, "Waste Technology Issues," undated.



TECHNICAL PERFORMANCE OF A GEOLOGIC REPOSITORY

- In essence, the performance of a geologic repository system boils down to a very detailed risk assessment of all of the physical and processes that could occur that may result in releases of radionuclides to the environment using predictions extrapolated to many thousands of years into the future.
- First, Scenario of Likely Events Must be Identified For the Chosen Repository Location (Yucca Mountain)
- Natural, High Probability
 - Natural Degradation of Engineered Barriers and Waste Forms
 - Movement of Radionuclides in Ground Water or Air
- Natural, Low Probability
 - Volcanism
 - Earthquakes
- Human Intrusion
 - Drilling
 - Mining

Source: S.A. Simonson, "Waste Technology Issues," undated.



SUB-SYSTEM INVESTIGATIONS

For Each of the Events, Predictive Models Must Be Developed Incorporating the Following Sub-System Models:

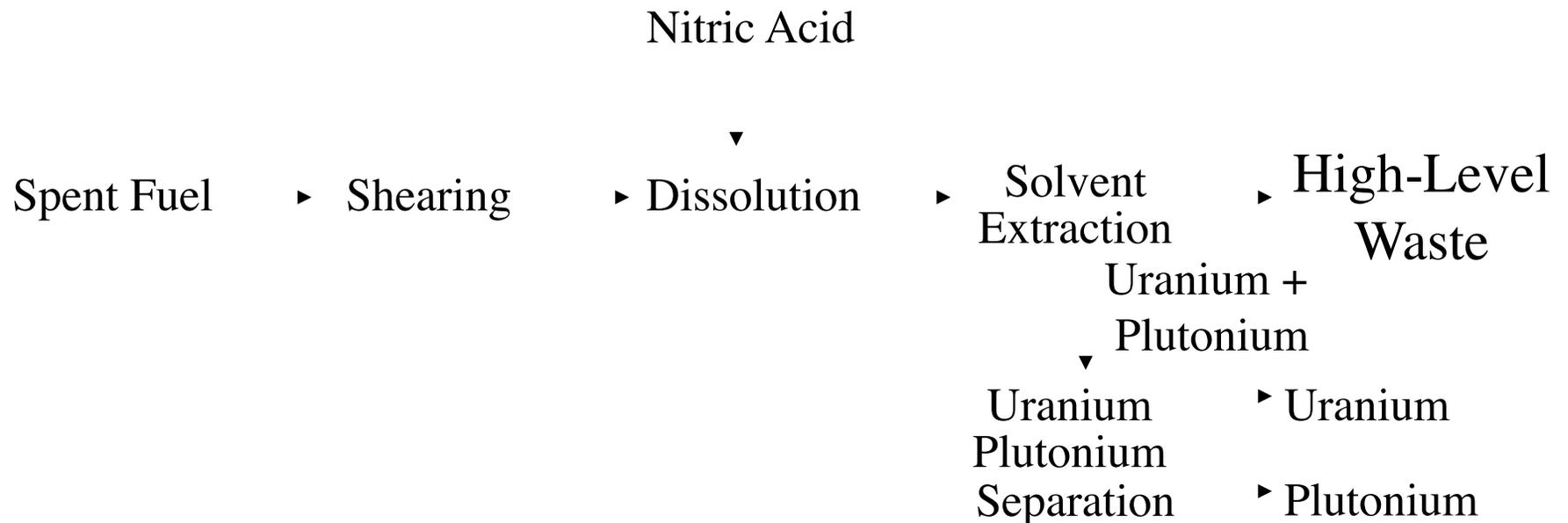
- Natural Barriers and Repository Influences
 - Radionuclide Transport in Ground Water
 - Radionuclide Transport as Vapors and Gases
 - Water Infiltration into Repository
- Engineered Barriers and Waste Forms



HIGH-LEVEL WASTE

DEFINITION: Wastes Arising from the Primary Decontamination Steps in the Reprocessing of Spent Fuel

PRUEX Process:

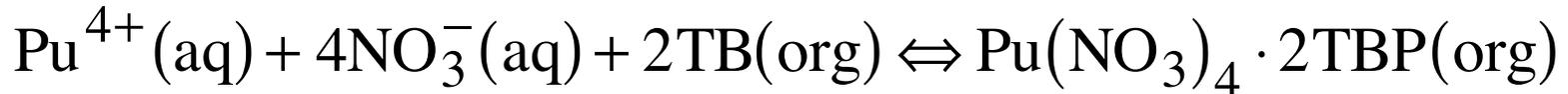
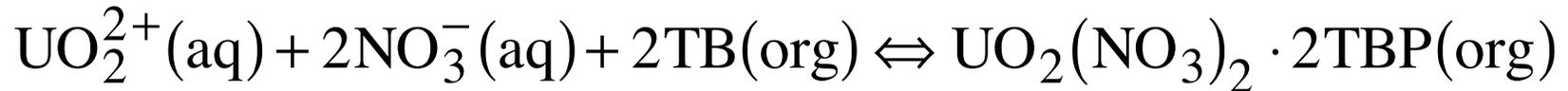


Source: S.A. Simonson, "Waste Technology Issues," undated.



TUTORIAL: SOLVENT EXTRACTION

Reaction:



Results in a Distribution of Uranium and Plutonium:

$$D_i \frac{\text{concentration of } i \text{ in organic phase}}{\text{concentration of } i \text{ in the aqueous phase}}$$

and a Net Separation of Uranium and Plutonium from the Fission Products:

$$\alpha = \frac{D_{\text{product}}}{D_{\text{impurity}}}$$

D's for Uranium and Plutonium are Much Higher Than the Fission Products, Thus a Separation (large α) is Made at Greater Than 99% in One Pass of Solvent Extraction.



REPROCESSING CONTINUED

After Separation, Uranium is Separated from the Plutonium by Chemically Changing the Aqueous Solution and Repeating the Solvent Extraction

Approximately 1% of the Spent Fuel is Plutonium of Which 70% is Fissile (7 g/kg spent fuel)

ECONOMICS:

Cost of Reprocessing	~\$1300/kg
Cost of Fuel Fabrication	~\$ 350/kg
Energy Value of Plutonium	~\$ 200/kg
Uranium Credit	~\$ 60/kg

Therefore, Marginal to Uneconomic to Reprocess at Current Facilities, Particularly When Uranium is Very Inexpensive

Fission Products and Actinide Wastes are Sent for Processing Into Glass Logs for Permanent Disposal

Source: S.A. Simonson, "Waste Technology Issues," undated.



WHY GLASS?

Historical Perspective of High-Level Waste Glass:

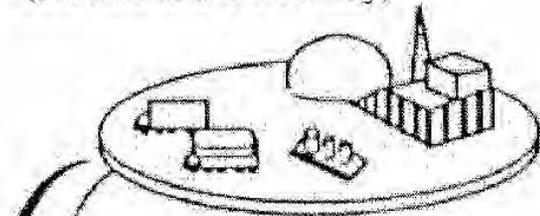
- Natural Analogs
- High Durability (10X better than spent fuel in retaining radionuclides)
- High Waste Loading (up to 30 wt% waste vs. 5% for spent fuel)
- Predictability of Degradation

Source: S.A. Simonson, "Waste Technology Issues," undated.

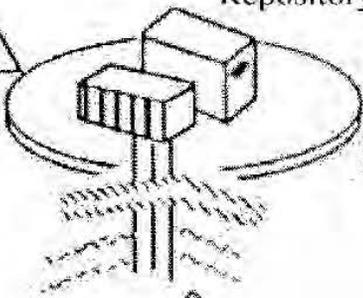


NUCLEAR WASTE MANAGEMENT SYSTEM

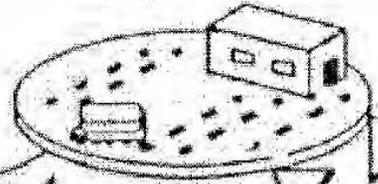
High-Level Waste Generators
(Defense and West Valley)



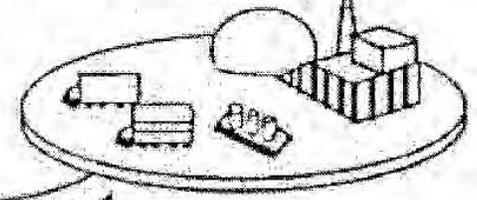
Repository



Monitored Retrievable
Storage



Most Reactors
(and Storage Sites)



After Repository
Completion

Nearby Reactors

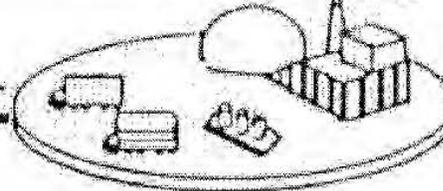
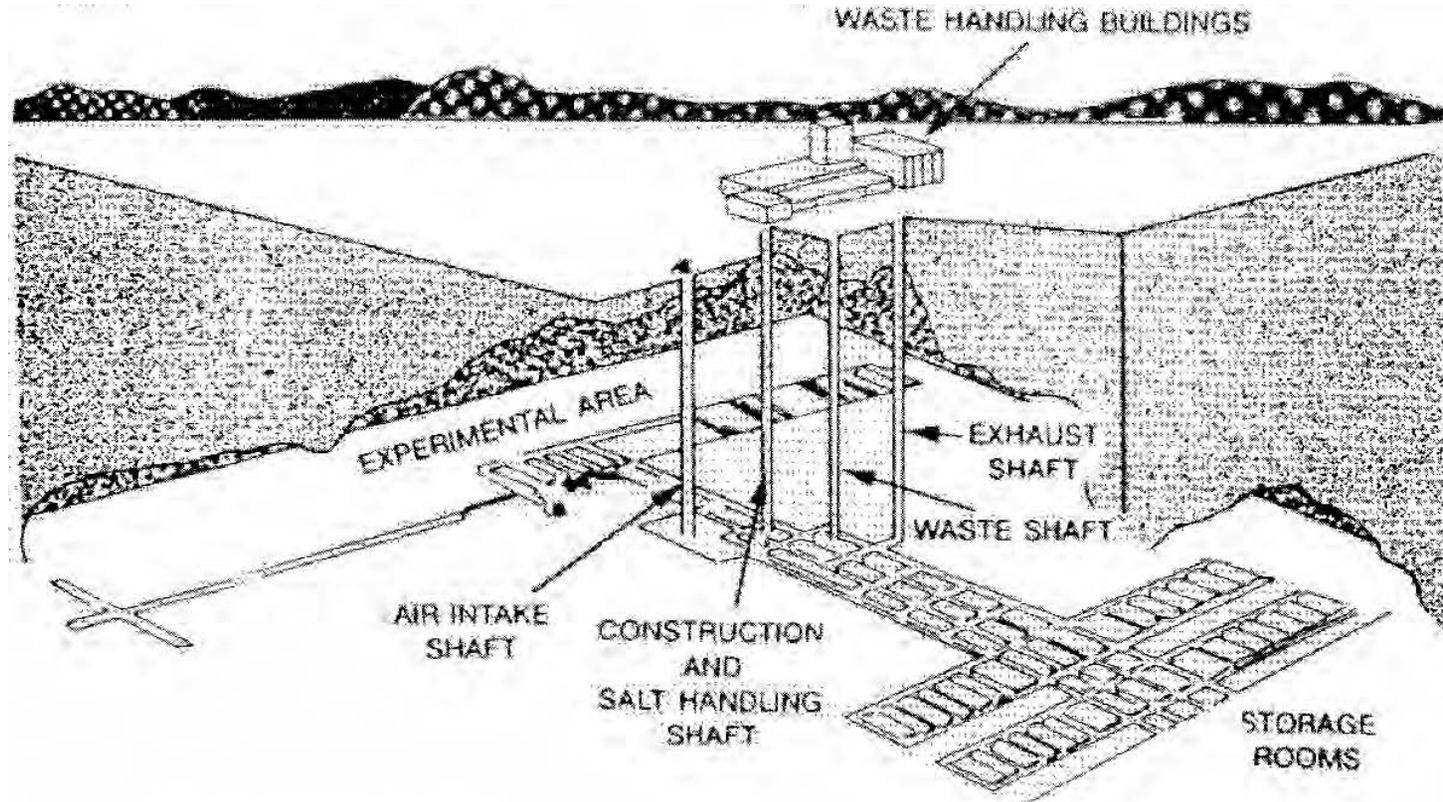


Image by U.S. Department of Energy.

Source: S.A. Simonson, "Waste Technology Issues," undated.



WASTE ISOLATION PILOT PLANT: ITS CAPACITY, ESTIMATED OPERATIONAL COST, AND ESTIMATED LIFETIME

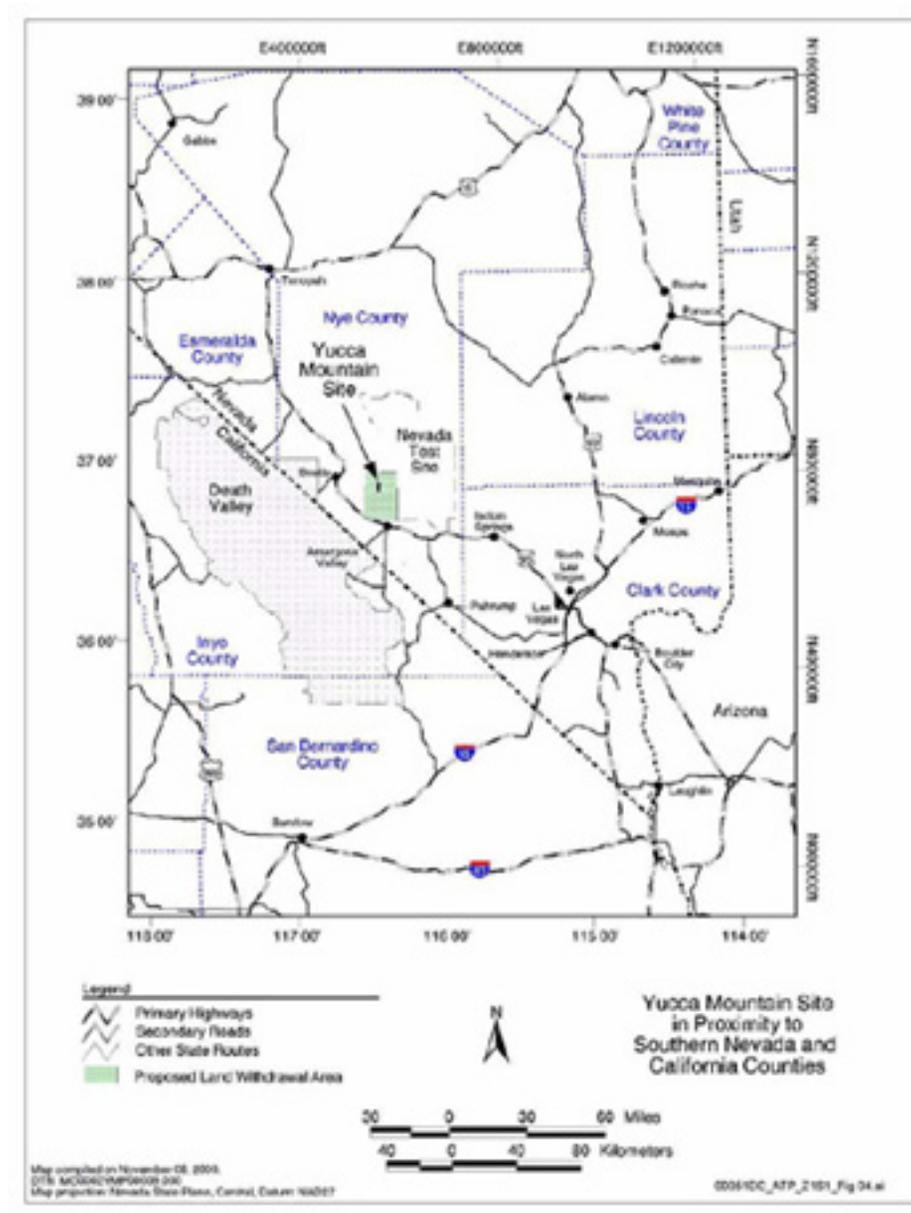


Total waste capacity:	6 million cubic feet
Current accumulated waste volume:	2.12 million cubic feet
Construction cost to date:	\$700 million
Estimated cost for 25-year operation:	\$2.5 billion
Estimated total time to fill the WIPP repository:	20 - 30 years

Fig. 2-3 in *Complex Cleanup: The Environmental Legacy of Nuclear Weapons Production*.
U.S. Congress, Office of Technology Assessment, February 1991, OTA-O-484.



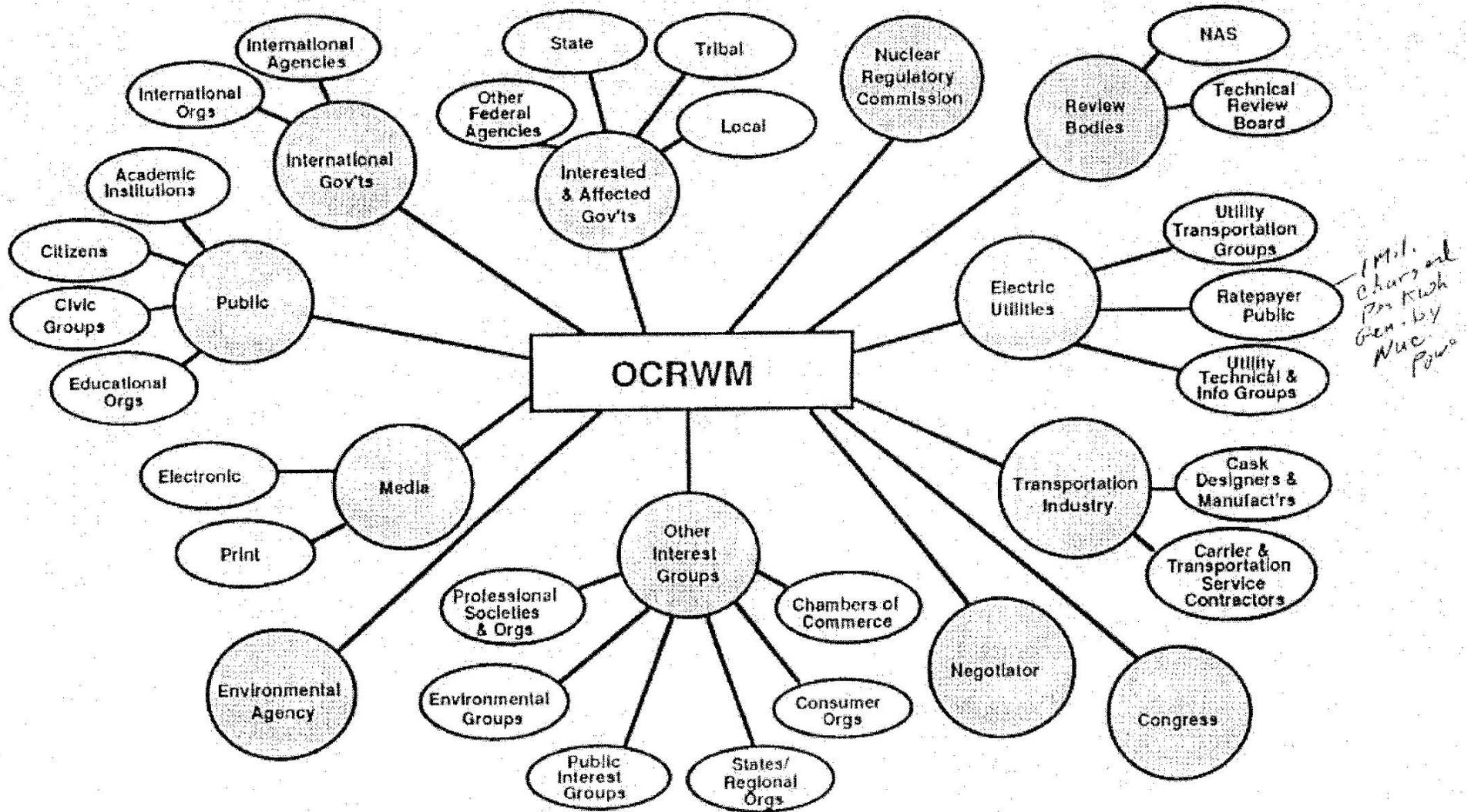
SOUTHERN NEVADA REGION



Source: Fig. 1-5 in "Yucca Mountain Science and Engineering Report." U.S. Department of Energy, Office of Civilian Radioactive Waste Management (February 2002): DOE/RW-0539-1.



INTERESTED PARTIES





OBSERVATIONS

- Complex, First-of-a-Kind Project
- Public Acceptance Dominates
- Numerous Oversight Entities
- Incredible Meetings Schedule
- Fire Drills Dominate Strategic Planning
- Radioactive Waste People are Competent and Hardworking
- DOE Bureaucracy is a Major Challenge

Source: S.A. Simonson, "Waste Technology Issues," undated.

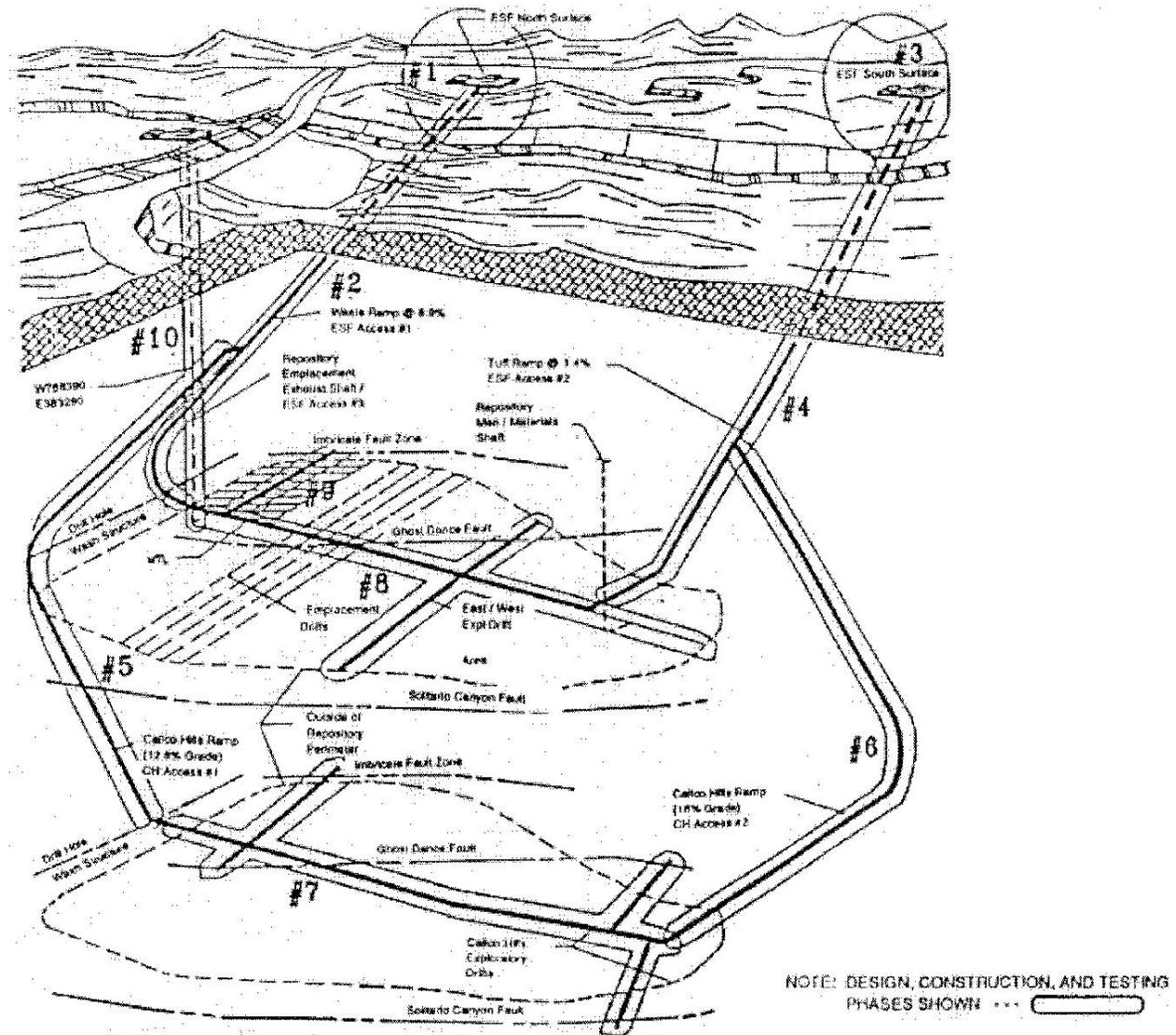


OBSERVATIONS (Continued)

- The M&O Welcomed by DOE/Community
- High Expectations
- Large, High Visibility Tasks Being Assigned
- M&O Identity and Team Integration is Good
- TRW/Team Identity and Reputation Will Be Applied
- M&O is Viewed as Different
 - Broadly capable
 - Mission/goal oriented
 - Experienced/up-to-speed
 - Not a support contractor



EXPLORATORY STUDIES FACILITY



NOTE: This is pictorial only and not drawn to scale.

Source: S.A. Simonson, "Waste Technology Issues," undated. 38

Slide 7 in Petrie, Edgar H. "Exploratory Shaft Facility Alternatives Study - Resumption of Design Activities." U.S. Department of Energy, Office of Civilian Radioactive Waste Management, March 7, 1991.



CANISTERS





YUCCA MOUNTAIN COUNTRYSIDE





YUCCA MOUNTAIN COUNTRYSIDE





YUCCA MOUNTAIN COUNTRYSIDE





YUCCAS AT YUCCA MOUNTAIN





YUCCA MOUNTAIN COUNTRYSIDE, METEOROLOGICAL STATION





YUCCA MOUNTAIN COUNTRYSIDE





YUCCA MOUNTAIN TUNNEL ENTRANCE, RAIL ENGINE





YUCCA MOUNTAIN EXCAVATION PILE





YUCCA MOUNTAIN ENTRANCE





YUCCA MOUNTAIN ENTRANCE





YUCCA MOUNTAIN TUNNEL





YUCCA MOUNTAIN TUNNEL





YUCCA MOUNTAIN TUNNEL





NPR IN ACTION





TUNNEL HEATING MEASUREMENT





TUNNEL HEATING MEASUREMENT, VISITING ENGINEER





TUNNEL HEATING MEASUREMENT



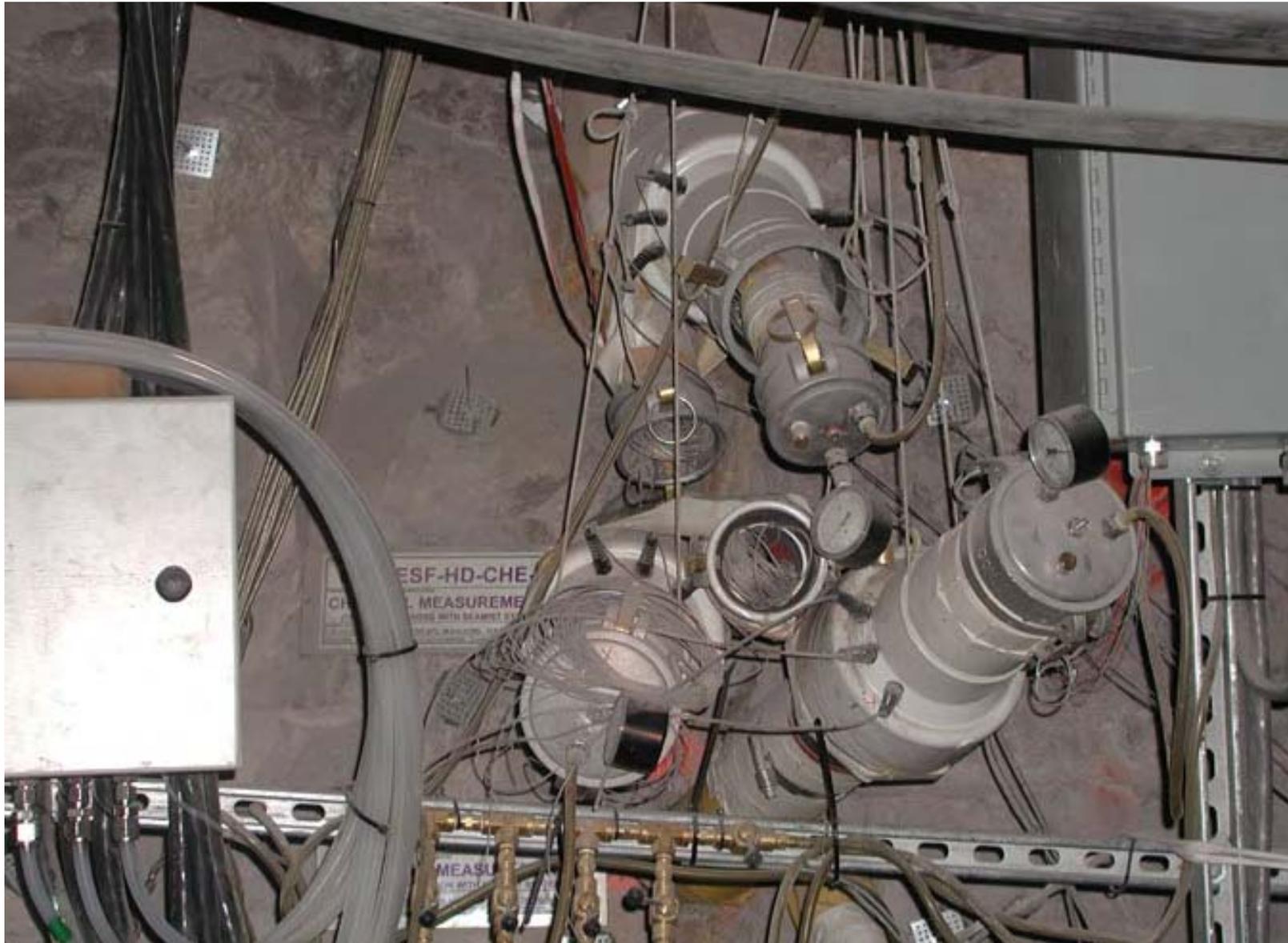


TUNNEL HEATING MEASUREMENT, THERMAL PROBES





TUNNEL HEATING MEASUREMENT, CHEMICAL PROBES





YUCCA MOUNTAIN WATER SUPPLY, SYMBOL OF FEDERAL-STATE RELATIONSHIP



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22.081J / 2.650J / 10.291J / 1.818J / 2.65J / 10.391J / 11.371J / 22.811J / ESD.166J

Introduction to Sustainable Energy

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